Decree
of the Nuclear Regulatory Authority of the Slovak Republic No. 30/2012 Coll. as amended by Decree No. 101/2016 Coll.
laying down details of requirements for managing the nuclear materials, radioactive waste and spent nuclear fuel (consolidated version)

The Nuclear Regulatory Authority of the Slovak Republic, pursuant to Section 12 (8) and Section 21(13) of Act No. 541/2004 Coll., on the peaceful use of nuclear energy (the Atomic Act), and on changes and amendments to some Acts (herein referred to as "Act") lays down as follows

Section 1
Scope of application

This decree lays down details of requirements for
a) handling of nuclear materials,
b) handling of spent nuclear fuel,
c) managing of the radioactive waste, including its generation,
d) classification of radioactive waste,
e) import and export of radioactive waste,
f) scope and contents of documentation when managing the radioactive waste,
g) facilities for management of radioactive waste,
h) record keeping for radioactive waste.

Section 2
General requirements for the handling of nuclear materials, spent nuclear fuel and managing radioactive waste

(1) When managing the radioactive waste and handling the spent nuclear fuel, interdependencies among individual steps during managing/handling are respected.
(2) Nuclear materials are handled so that
a) the effects of ionising radiation on workers, the population and the environment are kept at as low a level as can be reasonably achieved taking into account technical, economic and societal factors;
b) sub-criticality is ensured;
c) removal of residual heat is ensured;
d) the amount and activity of radioactive waste is kept at as low a level as can be reasonably achieved taking into account technical, economic and social factors;
e) mutual dependencies between individual steps in their handling are respected.
(3) The nuclear materials, radioactive waste and spent fuel shall be handled so as the undue technical, economic and social burden on future generations is not imposed.

Section 3
Requirements for managing the radioactive waste

(1) The management of radioactive waste, from its generation to its disposal, is documented in an accompanying sheet for radioactive waste.
(2) Radioactive waste is characterised through its physical, chemical and radiological properties for the purposes of its future handling as well as for its verification purpose, so that the characteristics of the radioactive waste are in accordance with the safety for its future management.
(3) To document and assess the management of radioactive waste, representative samples are taken and analysed at important management points by the authorisation holder accordingt o Section 5 (3) (f). These samples are kept until the radioactive waste has been placed in a

repository, and the samples from waste characterisation during repository operation are kept until the end of repository operation.
(4) Gaseous and liquid discharges are cleaned of radionuclides to as low as reasonable achievable level prior to being released into the environment.[1])
(5) In order to meet the requirements of Section 20 (9) of the Act, the authorisation holder shall establish a processing schedule for all radioactive waste that he produces, and shall ensure that it is followed and regularly assessed.

Section 4
Requirements for radioactive waste management facilities

(1) The safe and reliable operation of radioactive waste management facilities is ensured by the authorisation holder so that these facilities
a) allow the collection, storage and handling of radioactive waste;
b) are well-accessible, and conditions are established for their decontamination, checking, maintenance and repairs;
c) are designed to prevent their clogging as much as possible, and so that any deposits and sediments can be removed;
d) are resistant to mechanical, thermal, and corrosion damage;
e) are resistant to the effects of fire or explosion, if used for radioactive waste containing flammable or explosive substances.
(2) In radioactive waste management facilities, quantities are continuously or regularly measured, in order to prove the correct functioning of facilities or measurement of quantities with an influence on explosiveness or ignition of these facilities.
(3) Within nuclear facilities, liquid radioactive wastes are transported through piping. They are transported in packaging[2]) only if their transportation via piping is technically impossible or economically unfeasible.
(4) Authorisation holder pursuant to Section 5 (3) (b) to (d) of the Act is responsible for all safety aspects of the nuclear installation including radioactive waste managed therein. At each step of radioactive waste management, responsibility must be specified for radioactive wastes at the facility where they are managed, between the generator of the radioactive waste and the possessor of the radioactive waste.

Section 5
Radioactive waste classes

Based on activity, radioactive wastes are classified as follows:
a) transitional radioactive waste activity of which falls below the limit value for their release to the environment[1]) during storage
b) very low level radioactive waste, activity of which is slightly higher than the limit value for their release to the environment,[1]) contain mainly short lived radionuclides, or also a low concentration of long lived radionuclides, and which during storage require a lower degree of isolation from the environment through a system of engineered barriers, or don't require a system of engineered barrieres and the period of institutional control is shorter as in the case of near surface-type repositories;
c) low level radioactive waste, which average specific activity of long lived radionuclides, especially radionuclides emitting alpha radiation, is less than 400 Bq/g, maximum specific

[1)] Act of the Government of the Slovak Republic No. 345/2006 Coll., on basic safety requirements for the protection of health of workers and the population from ionising radiation. Act of the National Council of the Slovak Republic No. 355/2007 Coll., on the protection support and development of public health and on changes and amendments to some acts as amended.

[2)]Section 2 (b) of Decree of the Nuclear Regulatory Authority of the Slovak Republic No. 57/2006 Coll., laying down details of requirements for the transport of radioactive materials.

activity of long lived radionuclides, especially radionuclides emitting alpha radiation, is locally less than 4000 Bq/g, does not produce residual heat, and following conditioning shall meet the safe operating limits and conditions for near surface-type repositories;
d) intermediate level radioactive waste, which average specific activity of long lived radionuclides, especially radionuclides emitting alpha radiation, is equal to or over 400 Bq/g, may produce residual heat and measures for its removal are less than in the case of high level radioactive waste, and which following conditioning shall not meet safe operating limits and conditions for near surface-type repositories;
e) high level radioactive waste, whose average specific activity of long lived radionuclides, especially radionuclides emitting alpha radiation, exceeds values specified for low level and intermediate level radioactive waste requiring measures for the removal of residual heat and can be deposited only in an deep geological repository.

Section 6
Collection and sorting of radioactive waste

(1) The authorisation holder determines characteristic properties of radioactive wastes and specifies an appropriate sorting system in accordance with Section 5.
(2) Radioactive wastes are collected and sorted depending on the manner of their further management and with respect to their activity and physical, chemical, and biological properties.
(3) If technically feasible, collection and sorting is performed directly at the point where the radioactive waste is generated.
(4) Packaging designated for radioactive waste collection is labelled so that it is obvious what radioactive wastes it contains.

Section 7
Radioactive waste treatment

(1) Radioactive waste treatment is an activity focused on the separation of radionuclides from radioactive waste, on the modification of its composition, and on the reduction of its volume with the objective of increasing the safety and economic efficiency of its management.
(2) Radioactive wastes are treated so that utilisable substances are separated and returned for reuse and so that the amount of remaining radioactive waste is as low as possible taking into consideration its further management.
(3) If ion exchangers, filtration materials or separating materials with a limited useful life time are used in radioactive waste treatment, the authorisation holder shall regularly monitor their functionality and propose safe operational limits and conditions; when these have been reached, the materials must be regenerated or replaced.
(4) The authorisation holder shall determine the appropriate treatment system for all radioactive waste at least two alternatives and with a reasoned choice of one of them.

Section 8
Radioactive waste conditioning

(1) The result of radioactive waste conditioning is a waste package form of radioactive waste prepared in accordance with the requirements of safe handling, storage, transport and disposal. In its waste package form, radioactive waste may include packaging.
(2) The authorisation holder shall propose a technological procedure, conditions of radioactive waste conditioning, materials, and packaging used in radioactive waste conditioning.
(3) The technological procedure for radioactive waste conditioning includes a check of the required quality parameters of the waste package form, taking into consideration possible

influences caused by the properties of radioactive waste, materials and packaging, such as corrosion, production of gases, heat generation, volume instability and biodegradation.
(4) Approval of safe operational limits and conditions for radioactive waste conditioning and disposal facilities also include approval of waste package form.

Section 9
Radioactive waste storage

(1) Radioactive waste is stored separately from other waste or materials. For each radioactive waste storage facility, the authorisation holder shall determine the storage method, the maximum amount and activity of stored radioactive waste, as well as the expected date of its removal.
(2) A radioactive waste storage facility is designed and operated so that
a) during the expected storage period, it protects radioactive wastes from degradation due to the change of their characteristics as a result of negative climactic or meteoric conditions and and other external conditions;
b) storage safety is primarily ensured through passive safety elements;
c) it ensures that stored radioactive waste can be easily handled and withdrawn;
d) conditioned radioactive waste does not change properties determining its disposal.

(3) The capacity of a radioactive waste storage facility must correspond to related radioactive waste management technology with a sufficient reserve so that operation of such technology is not negatively affected.
(4) Creation of a reserve storage capacity margin for the purposes of relocating, repackaging, checking, maintaining, and withdrawing stored radioactive waste must already be taken into consideration during the radioactive waste storage facility's design phase. The radioactive waste storage facility operation authorisation shall ensure that reserve storage capacity is maintained throughout the entire period of the storage facility's operation.
(5) The authorisation holder shall reassess the adequacy of radioactive waste storage capacity every three years, taking into account the generation of radioactive waste, its expected storage period, and the availability of a suitable type of repository.
(6) The authorisation holder shall keep records of the precise locations of individual radioactive waste packages in storage.
(7) The radioactive waste storage facility's equipment shall correspond to the type, form, activity and amount of radioactive waste being stored. Storage premises shall be equipped with a ventilation and fire suppression system.[3])
(8) The holder of the authorisation for the operation of a nuclear installation or for the decommissioning stage of a nuclear installation shall determine, based on safety analyses, limits and conditions under which radioactive waste is to be stored, including internal storage conditions, criteria of radioactive waste’s acceptability for storage and measures to remove those radioactive wastes that, due to changes in their properties during storage, cannot be removed in a normal manner. The safety analyses include storage equipment, the type of radioactive waste stored and the corresponding relevant safety functions.
(9) The authorisation holder shall regularly monitor and assess the condition of the radioactive waste storage facility and the condition of stored radioactive waste, especially by checking the tightness of barriers and conditions inside the storage facility, and by monitoring dose rates and surface contamination in order to prove that safe storage facility operational limits and conditions are being met on a continuous basis.
(10) Tanks for the storage of liquid radioactive waste are
a) designed and operated to be watertight, protected from corrosion, and to the authorisation the homogenisation and emptying of their contents; every storage tank system always has free volume available as an emergency backup corresponding to the volume of the

[3]) Section 2 (1) (b) of the Act No. 314/2001 Coll. On fire protection as amended.

largest tank in the system; vapours from the tanks are collected and processed as radioactive waste;

b) protected against overflow, and their filling is monitored;

c) placed in protective leak-proof volumes with sufficient reserve to capture the contents of the tank, which are equipped with leak signalling and pumps; if liquid radioactive wastes are stored in vessels, the walls and floor of the storage facility are to be built leak-proof to such a height that they can safely contain the entire volume of the radioactive waste being stored.

Section 10
Dispoal of radioactive waste

(1) The authorisation holder shall only dispose into repositories waste package forms that comply with safe repository operational limits and conditions approved by the Authority based on safety analyses.

(2) Repository safety analyses represent a comprehensive assessment of risks related to the disposal of radioactive waste and proof of the functionality of the entire repository system from the perspective of its possible impact on humans and on the environment, taking into account the natural evolution of the repository and its possible intrusion during institutional control of the repository after its closure.

(3) The scope of repository safety analyses, the time interval assessed, input quantities and selection of their representative values and other parameters that limit safety analyses, with which their results will be compared and from which conditions of acceptability of radioactive waste for the repository will be derived, are to be proposed by the authorisation applicant or authorisation holder along with their respective justifications.

(4) Repository safety analyses include uncertainty analyses and analyses of the sensitivity of results to individual parameters.

(5) Safety analyses form a basis for decision-making on any changes or corrective measures in the repository and must demonstrate that a proposed change or corrective measure shall be optimal.

(6) During preparation of repository safety analyses, the period of institutional control after repository closure needed for uninterrupted preservation of its safety functions and for the procedure according to Section 22 (4) of the Act is to be taken into account.

(7) Institutional control may consist of an active part and a passive part.

(8) Characteristic properties of the repository, particularly the penetration of water into the repository and release of radioactive substances into the environment, are to be monitored throughout commissioning, operation and institutional control. It shall be ensured that the monitoring system does not compromise the repository's seals.

(9) A preliminary repository closure design, especially its stabilisation, coverage and building of a covering drainage system, is to be included in the pre-operational safety report.

Section 11
Requirements for import of radioactive waste

(1) The Authority shall decide whether to issue a radioactive waste import authorisation pursuant to Section 21(11) of the Act based on a written application of an individual or legal entity, which contains:

a) the type, amount, form, activity, and radionuclide composition of the radioactive waste to be imported or exported, along with a description of its physical, chemical, biological and other hazardous properties that could affect its planned management within the Slovak Republic; the activity of re-imported radioactive waste must be equivalent to the activity of the exported radioactive wastes, taking into consideration radioactive decay;

b) a description of all activities to be performed within the scope of import or export of the radioactive waste, including its delivery, receipt, transport, transfer and storage;

c) a description of proposed procedures for processing and treatment of imported or exported radioactive waste;
d) the name of the countries from which the radioactive waste is to be imported or to which it is to be exported;
e) the expected date of import and export of the radioactive waste;
f) confirmation that the applicant for a radioactive waste import authorisation has the authorisation for the transport and management of radioactive waste, or that he has contracted, organised and financially provided for a different individual or legal entity that has the authorisation for the transport and management of radioactive waste and that shall transport and handle the imported radioactive waste;
g) confirmation of approval of the re-export of imported radioactive waste following its processing;
h) proposed responsibilities for radioactive waste during its import, transport and management within the Slovak Republic and during its export;
i) a physical protection solution; [4])
j) proof that the safety level of nuclear facilities for the management of imported radioactive waste shall be maintained.
(2) Unless specified otherwise, responsibility for the handling of radioactive waste imported to the Slovak Republic is borne by the import authorisation holder.

Section 12
Radioactive waste management documentation

(1) Relevant safety reports submitted along with applications for a repository siting authorisation, a repository construction authorisation, a repository commissioning and the operating authorisation, the repository closure authorisation and for institutional control include a description and analysis of the safety function of barriers to prevent the spread of radioactive substances. The analysis contains a deterministic assessment of repository operational safety and a deterministic and probability assessment of long-term repository safety following its closure.
(2) Limits and conditions for safe operation of radioactive waste management facilities shall be determined on the basis of safety analyses, and shall include:
a) information on quantities characterising the conditions with which nuclear safety and radiation protection is ensured during the management of radioactive waste, and the manner and frequency of their measurement and assessment;
b) requirements for staff activities and for organisational measures leading to compliance with all defined conditions for designed operational states; as for the repository, also all the safety requirements after the repository closure in a manner they were considered in the safety analyses,
c) requirements for procedures, frequency, type and scope of radioactive waste checks performed to demonstrate compliance with limits and conditions as well as waste acceptance criteria;
d) measures for management of radioactive waste that does not meet waste acceptance criteria.
(3) Safe storage operating limits and conditions include conditions of acceptability of radioactive waste for storage, which must be in accordance with requirements for further management of stored radioactive waste.
(4) Safe repository operating limits and conditions include conditions of acceptability of waste package forms for the repository, particularly the type of waste package form and its

[4]) Decree of the Nuclear Regulatory Authority of the Slovak Republic No. 51/2006 Coll., Laying Down Details of Requirements for Provisions of Physical Protection.

structural stability, leachability, thermal and radiation effects, possibility of a critical condition or microbial degradation occurring, gas formation, content of corrosive, explosive and self-igniting substances, flammable materials, free liquids and complexing or chelating agents, surface contamination, dose rate, dimensions, weight, and labelling of the packaged form of radioactive waste.

Section 13
Radioactive waste management recordkeeping

(1) The authorisation holder pursuant to Section 5 (3) (b) to (d) of the Act shall keep records of radioactive waste management. He shall hand over the records along with the radioactive waste. The holder of the repository operation authorisation shall keep these records from receipt of radioactive waste until closure of the repository.
(2) Records pursuant to Paragraph 1 contain:
a) accompanying sheets of generated or received and handed over radioactive waste;
b) records of the manner and course of radioactive waste management, including information pursuant to Section 9 (6);
c) records of radioactive waste sample analyses pursuant to Section 3 (3).
(3) A holder of the authorisation for the commissioning and operation of a nuclear installation for the management of radioactive waste records information pursuant to a special decree[5]) and keeps these records until authorisation termination.
(4) The holder of the authorisation for the closure of a repository and for institutional control records information pursuant to a special decree[5]) and keeps it until the end of institutional control period.

Section 14
Accompanying sheets for radioactive waste

(1) During each delivery and receipt of radioactive waste, the individual or legal entity making the delivery issues radioactive waste accompanying sheets based on the characterisation of the radioactive waste.
(2) The scope of characterisation of radioactive waste is based on conditions of the radioactive waste's acceptability for disposal or storage, if it does not meet disposal conditions.
(3) The authorisation holder must have a system and procedures in place to characterise the radioactive waste he manages.
(4) The individual or legal entity that is taking receipt of radioactive waste verifies that the individual or legal entity that is handing over the radioactive waste has characterised the radioactive waste correctly.
(5) The radioactive waste receipt and handover process is included in the nuclear installation's quality management system documentation and quality requirements.
(6) Radioactive waste accompanying sheets contain:
a) specifications characterising their form and origin;
b) the type and identification label of the packaging;
c) the start and end date of package filling;
d) the type and identification label of the waste package form;
e) overall activity of alpha and beta radionuclides and how it is documented;
f) the activity of individual radionuclides, the content of which is limited by specified acceptance criteria, and how it is documented;
g) the activity of other important radionuclides whose share of overall activity exceeds 1%, and how it is documented;

[5]) Decree of the Nuclear Regulatory Authority of the Slovak Republic No. 430/2011 Coll. on nuclear safety requirements.

h) values of leachability, compressive strength, hazardous substance content and other parameters determined from safety analyses of radioactive waste treatment, conditioning, storage, transportation and disposal;
i) a description of heterogeneities present in the packaged form with information pursuant to items e) to h);
j) the dose rate on the surface of the packaging;
k) the total weight of the filled packaging;
l) information on radionuclide contamination of the packaging surface;
m) the date the accompanying sheet was issued, indicating the documentation on which basis it was issued;
n) the business name and identification number of the delivering and receiving individual or legal entity, and the name, surname, position and signature of its authorised employee.

Section 15
Requirements for the handling of spent nuclear fuel

(1) General requirements for the handling of spent nuclear fuel shall be reflected in:
a) the design documentation of a nuclear installation where spent nuclear fuel is handled;
b) written organisational instructions for safe operation of the nuclear installation.
(2) Limits and conditions for safe operation of a nuclear installation shall be stipulated on the basis of a safety report, and include
a) information on quantities that characterise conditions under which nuclear safety and radiation protection are ensured during the handling of spent nuclear fuel, and the manner and periodicity of their measurement and evaluation;
b) requirements for staff activity and organisational measures leading to the fulfilment of all defined conditions for design operating conditions, as taken into account in the safety report;
c) requirements for procedures, frequency, type, and scope of inspections of spent nuclear fuel conducted in order to demonstrate compliance with limits and conditions under acceptability criteria in the safety report;
d) measures for handling spent nuclear fuel that fails to meet acceptability criteria arising from the safety report.
(3) Safety documentation for handling of spent nuclear fuel is governed, where appropriate, according to the provisions of Section 12.
(4) Records are kept of the handling of spent nuclear fuel, which contain:
a) information identifying the spent nuclear fuel,
b) its nuclear reactor irradiation history,
c) the isotopic composition of the spent nuclear fuel following its removal from the nuclear reactor,
d) the location of the spent nuclear fuel,
e) information on the tightness of the spent nuclear fuel's cladding,
f) information listed in approved safe operation limits and conditions.

Section 16
Storage of spent nuclear fuel

(1) Spent nuclear fuel is stored separately from other waste or materials.
(2) The holder of the authorisation to handle spent nuclear fuel stipulates, for each related nuclear installation, the manner of storage, the maximum amount, and the activity of stored spent fuel, as well as the expected date it shall be withdrawn.
(3) A spent nuclear fuel storage facility serves for the safe keeping of spent nuclear fuel prior to its reprocessing or deposit.
(4) The design and construction of a spent nuclear fuel storage facility must prove:

a) at least 5% subcriticality to be ensured in all operational conditions, and 2% during operational events, this being either through appropriate arrangement of spent nuclear fuel or through placement of a solid neutron absorber in the storage premises; the efficiency of the solid absorber shall be proven by calculation or experiment;
b) permanent removal of residual heat produced by the spent nuclear fuel from the storage premises; removal of heat shall be ensured through natural or forced circulation of a coolant so that the temperature of any part of the spent nuclear fuel cladding does not exceed the limit value;
c) its full or partial decontamination;
d) safe and proper handling and withdrawal of stored spent nuclear fuel in the usual manner during the entire duration of its storage;
e) recordkeeping and checking of stored spent nuclear fuel in accordance with special decree; [6])
f) adequate physical protection of the storage premises in accordance with provisions of special decree; [7])
g) prevention of falls of heavy objects into the spent nuclear fuel storage premises;
h) effective cleaning, replenishment and collection of cooling media leaks in wet spent nuclear fuel storage facilities;
i) effective ventilation of storage premises, preventing build-up of radioactive gases;
j) radiation protection in accordance with special act;[1])
k) provision of a permanent source of power for maintaining the activity of important operational and safety systems;
l) monitoring of the environmental impact of the spent nuclear storage facility's operation;
m) protecting spent nuclear fuel during its expected storage period from degradation due to changes in its properties as a result of negative conditions within the storage facility or weather or other outdoor conditions,
n) preferentially ensuring storage safety using passive safety elements
o) ensuring such conditions under which the stored spent fuel does not change its properties determining its disposal,
p) decommissioning of the spent nuclear storage facility after its operation has ended
q) safe handling of spent nuclear fuel following the end of its storage.

(5) Creation of reserve storage capacity for relocation of stored spent nuclear fuel; the transfer, checking, maintenance and withdrawal thereof is already taken into account during the spent nuclear fuel storage facility's design phase. The holder of a spent nuclear fuel storage facility operational authorisation shall ensure that reserve storage capacity is maintained during the entire time the nuclear installation's storage facility is in operation.

(6) The authorisation holder shall regularly monitor and assess the condition of the spent nuclear fuel storage facility and of the stored spent nuclear fuel, especially checking for leaks in barriers, internal storage facility conditions, and monitoring dose rates and surface contamination in order to demonstrate continuous compliance with limits and conditions for safe storage facility operation.

(7) The holder of a spent nuclear fuel storage facility operational authorisation stipulates, based on safety analyses, the limits and conditions under which spent nuclear fuel is stored, including indoor storage conditions, spent nuclear fuel acceptability criteria, and measures to make such spent nuclear fuel fit for withdrawal if it cannot be withdrawn in the usual manner due to changes to its properties during storage. Safety analyses cover storage facility equipment, the type of spent nuclear fuel being stored, and ensuing relevant safety functions.

(8) Calculations for facilities for the storage or transport of spent nuclear fuel may take into account the real isotopic composition of the spent nuclear fuel, the structural materials of

[6]) Decree of the Nuclear Regulatory Authority of the Slovak Republic No. 54/2006 Coll., on Accountancy for and Control of Nuclear Materials and on Notification of Selected Activities.
[7]) Decree No. 51/2006 Coll.

storage or transport facilities, and the cooling medium when specifying boundary conditions for calculation.

Section 17
Reprocessing of spent nuclear fuel

(1) The reprocessing of spent nuclear fuel is a technological procedure used to obtain nuclear materials suitable for further use through their chemical separation from fission products and other radionuclides in the spent nuclear fuel.
(2) Fission products and radionuclides separated during spent nuclear fuel reprocessing create radioactive waste that is modified into a state suitable for storage or deposit in a repository.
(3) The design of the spent nuclear fuel reprocessing nuclear installation and handling of nuclear materials in the course of reprocessing are subject, as appropriate, to the provisions of Section 16 (4).
(4) The handling of radioactive waste from spent nuclear fuel reprocessing is subject, as appropriate, to the provisions of Section (2) to (14).

Section 18
Deposition of spent nuclear fuel

(1) Spent nuclear fuel is deposited in a repository.
(2) Repositories are governed, as appropriate, by requirements listed in Section 10 and Section 16.
(3) Only spent nuclear fuel modified to meet the requirements of Section 21 (3) of the Act can be deposited in a repository without the repository operator intervention.

Section 19
Requirements for the handling of nuclear materials

The handling of nuclear materials is subject, where appropriate, to the provisions of Sections 3 to 9 and Sections 15 and 16, and the authorisation holder shall ensure:
a) that the nuclear material is not used for purposes in contravention of international treaties by which the Slovak Republic is a party to,[8]) and shall not be used to achieve any military objectives;
b) the physical protection of nuclear materials pursuant to special decree;[7])
c) that the nuclear material will not be provided to another individual or legal entity without the Authority's consent.

Section 20

This Decree has been adopted in accordance with a legally binding Act of the European Union in the area of technical standards and technical regulations. [9])

Section 21

[8])E.g. decree of the Ministry of foreign ffairs No. 61/1974 Coll. On Treaty on the Non-Proliferation of Nuclear Weapons, deecree of the Ministry of foreigh affairs No. 62/1974 Coll. On Treaty on the prohibition of the emplacement of the nuclear weapons and other wepons of mass destruction on the seabed and the ocean floor and in the subsoil thereof.
[9]) Directive 98/34/EC of the European Parliament and of the Council of 22 June 1998 laying down a procedure for the provision of information in the field of technical standards and regulations (OJ L 204, 21.7.1998, p. 37–48) as amended.

Decree of the Nuclear Regulatory Authority of the Slovak Republic No. 53/2006 Coll. laying down details of requirements for the handling of nuclear materials, radioactive waste and spent nuclear fuel, shall be repealed.

Section 22
Entry into force

This decree shall enter into force on 1 March 2012.

Marta Žiaková, m.p.